

CLIPPEDIMAGE= EP000347255A1

PUB-NO: EP000347255A1

DOCUMENT-IDENTIFIER: EP 347255 A1

TITLE: Method of treating high-level radioactive waste liquid.

PUBN-DATE: December 20, 1989

INVENTOR-INFORMATION:

NAME	COUNTRY
OHTSUKA, KATSUYUKI DORYOKURO KA	N/A
KONDOKU, ISAO DORYOKURO KAKUNENR	N/A
KAWASAKI, TAKESHI DORYOKURO KAK	N/A

ASSIGNEE-INFORMATION:

NAME	COUNTRY
DORYOKURO KAKUNENRYO	JP

APPL-NO: EP89306156

APPL-DATE: June 16, 1989

PRIORITY-DATA: JP14965488A (June 17, 1988)

INT-CL\_(IPC): G21F009/08

EUR-CL\_(EPC): G21F009/08

US-CL-CURRENT: 588/20

ABSTRACT:

A method of treating a high-level radioactive waste liquid comprises a freezing and drying step wherein condensate, a nitric acid solution, and bulk wastes mainly containing sodium nitrate, sodium hydroxide and the like are separated from a high-level radioactive waste liquid, and most fission products, actinoids, and corrosion products occurring in a reprocessing process, including iron, chromium, nickel, etc., are separated as a residue containing nitrate, oxides, and the like in a safe form. A sodium hydroxide solution or the like is added to that residue to dissolve the salts of sodium nitrate, sodium hydroxide, and the like, and the residue that is not dissolved in the solution, including fission products and the corrosion products

occurring in  
the reprocessing process, is separated. Thus, the recovery of  
useful elements  
contained in the undissolved residue is facilitated. As compared  
with a  
conventional case where the high-level radioactive waste liquid  
is glassified  
as it is, it is possible to substantially reduce the amount of  
substances  
glassified, and it is possible to enhance safety without causing  
corrosion of  
the material of equipment, an explosion, a fire, or the like.



Europäisches Patentamt  
European Patent Office  
Office européen des brevets

⑪ Publication number:

0 347 255  
A1

⑫

## EUROPEAN PATENT APPLICATION

⑬ Application number: 89306156.4

⑮ Int. Cl.<sup>4</sup>: G 21 F 9/08

⑭ Date of filing: 16.06.89

⑩ Priority: 17.06.88 JP 149654/88

⑯ Date of publication of application:  
20.12.89 Bulletin 89/51

⑰ Designated Contracting States: DE FR GB

⑲ Applicant: DORYOKURO KAKUNENRYO KAIHATSU  
JIGYODAN  
9-13, Akasaka 1-chome  
Minato-ku Tokyo 107 (JP)

⑳ Inventor: Ohtsuka, Katsuyuki Doryokuro Kakunenryo  
Kaihatsu  
Jigyodan Tokai Jigyosho 4-33, Oaza-Muramatsu  
Tokai-mura Naka-gun Ibaraki 319-11 (JP)

Kondoh, Isao Doryokuro Kakunenryo Kaihatsu  
Jigyodan Tokai Jigyosho 4-33, Oaza-Muramatsu  
Tokai-mura Naka-gun Ibaraki 319-11 (JP)

Kawasaki, Takeshi Doryokuro Kakunenryo Kaihatsu  
Jigyodan Tokai Jigyosho 4-33, Oaza-Muramatsu  
Tokai-mura Naka-gun Ibaraki 319-11 (JP)

㉑ Representative: Bubb, Antony John Allen et al  
GEE & CO. Chancery House Chancery Lane  
London WC2A 1QU (GB)

㉒ Method of treating high-level radioactive waste liquid.

㉓ A method of treating a high-level radioactive waste liquid comprises a freezing and drying step wherein condensate, a nitric acid solution, and bulk wastes mainly containing sodium nitrate, sodium hydroxide and the like are separated from a high-level radioactive waste liquid, and most fission products, actinoids, and corrosion products occurring in a reprocessing process, including iron, chromium, nickel, etc., are separated as a residue containing nitrate, oxides, and the like in a safe form. A sodium hydroxide solution or the like is added to that residue to dissolve the salts of sodium nitrate, sodium hydroxide, and the like, and the residue that is not dissolved in the solution, including fission products and the corrosion products occurring in the reprocessing process, is separated. Thus, the recovery of useful elements contained in the undissolved residue is facilitated. As compared with a conventional case where the high-level radioactive waste liquid is classified as it is, it is possible to substantially reduce the amount of substances classified, and it is possible to enhance safety without causing corrosion of the material of equipment, an explosion, a fire, or the like.

EP 0 347 255 A1

**Description****METHOD OF TREATING HIGH-LEVEL RADIOACTIVE WASTE LIQUID**

The present invention relates to a method of treating a high-level radioactive waste liquid produced in a reprocessing plant. More specifically, the present invention concerns a method of treating a high-level radioactive waste liquid wherein the high-level radioactive waste liquid is largely classified into (1) condensate, (2) bulk wastes mainly containing sodium nitrate, sodium hydroxide, etc., and (3) a residue comprising fission products, actinoids, and corrosion products (iron, chromium, nickel, etc.) occurring in a reprocessing process, including nitrate, oxides, etc., and wherein (1) the condensate and (2) sodium nitrate, sodium hydroxide, etc., are converted into a low-level radioactive waste, and the amount of the residue including fission products, actinoids, and corrosion products is reduced substantially by removing (3) sodium nitrate, sodium hydroxide, etc. In this method, as the residue is stored in the form of nitrate, oxides, and the like, storage is facilitated, and recovery of useful elements in the future is made possible as the residue containing useful elements is separated.

Hitherto, as a method of permanently separating radioactive wastes from an environment without requiring artificial operations, a glassification treatment process is adopted in which waste is provisionally stored as a liquid for a certain period and is then glassified to ensure that the radioactive substances contained in the waste will not move for a long period of time. The radioactive waste liquid produced in a reprocessing plant is mainly constituted by a large amount of sodium nitrate and sodium hydroxide, and disposal of this waste liquid is effected by dissolving the waste together with a large amount of a glass-forming agent, thereby effecting the aforementioned glassification treatment.

However, when glass having good qualities (properties) is formed in this glassification treatment, the amount of sodium that can be contained in the glass is limited. Consequently, there is a drawback in that it is difficult to convert a large amount of high-level waste liquid into a small amount of a glassified body, with the result that the amount of glassified body produced becomes disadvantageously large.

In addition, since the high-level radioactive liquid is heated and dissolved at a high temperature, there is the problem of the component materials of equipment from becoming corroded. Hence, when a need arises in the future to recover useful elements, the separation and recovery of useful elements are estimated to be difficult.

Accordingly, an object of the present invention is to provide a method of treating a high-level radioactive waste liquid which is (1) capable of safely separating from a high-level radioactive waste liquid a condensate, a nitric acid solution, and bulk wastes mainly containing sodium nitrate, sodium hydroxide, etc., and of separating most of fission products, actinoids, and corrosion products occurring during a reprocessing process such as iron, chromium,

nickel, etc., as a residue containing nitrate, oxides, etc. in a stable form, which (2) facilitates recovery of useful elements contained in the residue, which is (3) capable of substantially reducing the volume of a glassified body as compared with a conventional case where the high-level radioactive waste liquid is glassified as it is, and which is (4) capable of enhancing safety without causing corrosion of the materials of equipment, an explosion, a fire, or the like.

To this end, according to the present invention, there is provided a method of treating a high-level radioactive waste liquid, comprising the steps of: freezing and sublimating a high-level radioactive waste liquid produced at a reprocessing plant so as to separate the high-level radioactive waste liquid into evaporation substances that are sublimated and the residue that is not sublimated; and condensing the sublimated evaporation substances; and adding a solution to the residue that is not sublimated and separating the mixture into a solid and a liquid, the solid being the residue that is not dissolved in the solution, and the liquid being a solution in which the other components are dissolved.

More specifically, in the condensing step, a low-level radioactive waste liquid mainly containing water, nitrate, and nuclides that are sublimated is extracted as the condensate. The residue that is not sublimated mainly comprises salts of sodium nitrate and sodium hydroxide, fission products, and corrosion products occurring in a reprocessing process. In the solid/liquid separating step, a sodium hydroxide solution or the like is added to the residue that is not sublimated to dissolve the salts of sodium nitrate, sodium hydroxide, and the like, and the mixture is separated into a solution and the residue that is not dissolved in the solution, including fission products and corrosion products and the like occurring in a reprocessing process. The residue that is not dissolved is stored in the form of nitrate, hydroxides, a roasted body, or the like. The solution separated in the solid-liquid separating step is treated in a low-level radioactive waste liquid treating system.

**BRIEF DESCRIPTION OF THE DRAWINGS**

Fig. 1 is a schematic diagram of the overall configuration of a method of treating a high-level radioactive waste liquid in accordance with the present invention; and

Fig. 2 is a diagram illustrating process flow thereof.

Referring now to the accompanying drawings, a description will be given of the preferred embodiments of the present invention.

Fig. 1 is a schematic diagram of the overall configuration of a method of treating a high-level radioactive waste liquid in accordance with the present invention. In the drawings, reference numeral 1 denotes a freezing and drying apparatus; 2, a

high-level radioactive waste liquid; 3, a refrigerant; 4, a heating fluid (medium); 5, a condenser; 6, a vacuum pump; 7, a refrigerant; and 8, a condensate-receiving tank.

In the drawing, the high-level radioactive waste liquid 2 is adapted to be supplied from a high-level radioactive waste liquid-supplying system and a solution-supplying system to the freezing and drying apparatus 1. The refrigerant 3 and the heating medium 4 are supplied to the freezing and drying apparatus 1 and are discharged therefrom so as to cool and freeze, sublimate and dry the high-level radioactive waste liquid 2. The substance sublimated in the freezing and drying apparatus is supplied to the condenser 5. The refrigerant 7 is supplied to the condenser 5 and is discharged therefrom, and is exhausted by the vacuum pump 6. The condensate is stored in the condensate-receiving tank 8, and is removed therefrom.

Referring now to Fig. 2, a description will be given of the process flow of the high-level radioactive waste liquid.

The high-level radioactive waste liquid ① containing sodium nitrate, sodium hydroxide, fission products, actinoids, corrosion products (iron, chromium, nickel, etc.), during a reprocessing process and the like is cooled and frozen in a freezing process ②. The frozen high-level radioactive waste liquid (frozen material) is subjected to heating and pressure reduction and is placed in a vacuum in a sublimation process ③. Hence, nitric acid, water, nuclides, etc., are sublimated from the frozen material, and the sublimated nitric acid, water, nuclides, etc., are condensed into condensate ⑥ in the condenser. In the main, sodium nitrate and sodium hydroxide, as well as fission products, actinoids, and corrosion products (iron, chromium, nickel, etc.) occurring the reprocessing process and all of which do not evaporate, remain as the residue ④ of nitrate. If, for instance, a sodium hydroxide solution is added to this residue so as to dissolve sodium nitrate and sodium hydroxide, most of the fission products, actinoids, and the corrosion products occurring during the reprocessing process are not dissolved and remain as the residue. Next, in a solid/liquid separation process ⑤, the waste liquid is separated into a solution ⑦ containing sodium nitrate, sodium hydroxide and the like and into the residue ⑧ mainly consisting of the fission products, actinoids, and the corrosion products in the reprocessing process.

The solution ⑦ containing sodium nitrate, sodium hydroxide and the like is sent to a low-level radioactive waste liquid processing system ⑨ and is either refined and reused or used as raw material for low-level radioactive waste solids. The residue ⑧ is dried and roasted in a process 11, for instance, so as to be provided with the form and volume of nitrate, dried and roasted products, or the like for facilitating storage. In addition, since its form is suited for separating and recovering useful elements contained through cupellation or the like in the future, it is sent to a storage system.

As described above, in accordance with the present invention, the high-level radioactive waste

liquid can be separated into (a) condensate (nitric acid solution), (b) bulk wastes mainly containing sodium nitrate, sodium hydroxide, etc., and (c) residue (nitrate) mainly containing fission products, actinoids, and corrosion products (iron, chromium, nickel, etc.) in the reprocessing process. As a result, (1) since the high-level radioactive waste liquid is separated by the freeze-vacuum-drying process, (2) the problem of corrosion due to high temperature is overcome, (3) and the condensate (nitric acid solution) and bulk wastes mainly containing sodium nitrate and sodium hydroxide can be converted into low-level radioactive waste. In addition, (4) if the residue containing fission products, actinoids, and corrosion products (iron, chromium, nickel, etc.) that have stable configurations as nitrate and roasted product is stored in the form of residue, it is advantageous at the time when useful elements (rhodium, palladium, etc.) contained in the residue (nitrate or roasted material) are recovered in the future. Furthermore, (5) when a glassified body is made, since the amount of sodium is small, a high-quality glassified body can be formed, so that the amount of a glassified body produced can be reduced substantially. Moreover, fractionated substances can be used as raw material for artificial minerals (e.g., titanium, zirconium, calcium, barium, aluminum).

### 30 Claims

1. A method of treating a high-level radioactive waste liquid, comprising the steps of freezing and sublimating a high-level radioactive waste liquid produced at a reprocessing plant so as to separate said high-level radioactive waste liquid into evaporation substances that are sublimated and the residue that is not sublimated; condensing the sublimated evaporation substances; adding a solution to the residue that is not sublimated and separating the mixture into a solid and a liquid, said solid being the residue that is not dissolved in said solution, and said liquid being a solution in which the other components are dissolved.
2. A method according to Claim 1, wherein, in said condensing step, a low-level radioactive waste liquid mainly containing water, nitrate, and nuclides that are sublimated is extracted as the condensate.
3. A method according to Claim 1 or 2, wherein said residue that is not sublimated mainly comprises salts of sodium nitrate and sodium hydroxide, fission products, and corrosion products occurring in a reprocessing process.
4. A method according to any one of Claims 1 - 3, wherein, in said solid/liquid separating step, a sodium hydroxide solution or the like is added as said solution to the residue that is not sublimated to dissolve salts of sodium nitrate, sodium hydroxide, and the like, and the mixture is separated into a solution and the residue that is not dissolved in the solution, the latter

including fission products and corrosion products and the like occurring in a reprocessing process.

5. A method according to any one of Claims 1 - 4, wherein said residue that is not dissolved is stored in the form of nitrate, hydroxides, a

roasted body, or the like.

6. A method as claimed in any one of Claims 1 - 5, wherein the solution separated in said solid-liquid separating step is treated in a low-level radioactive waste liquid treating system.

5

10

15

20

25

30

35

40

45

50

55

60

65

FIG. 1

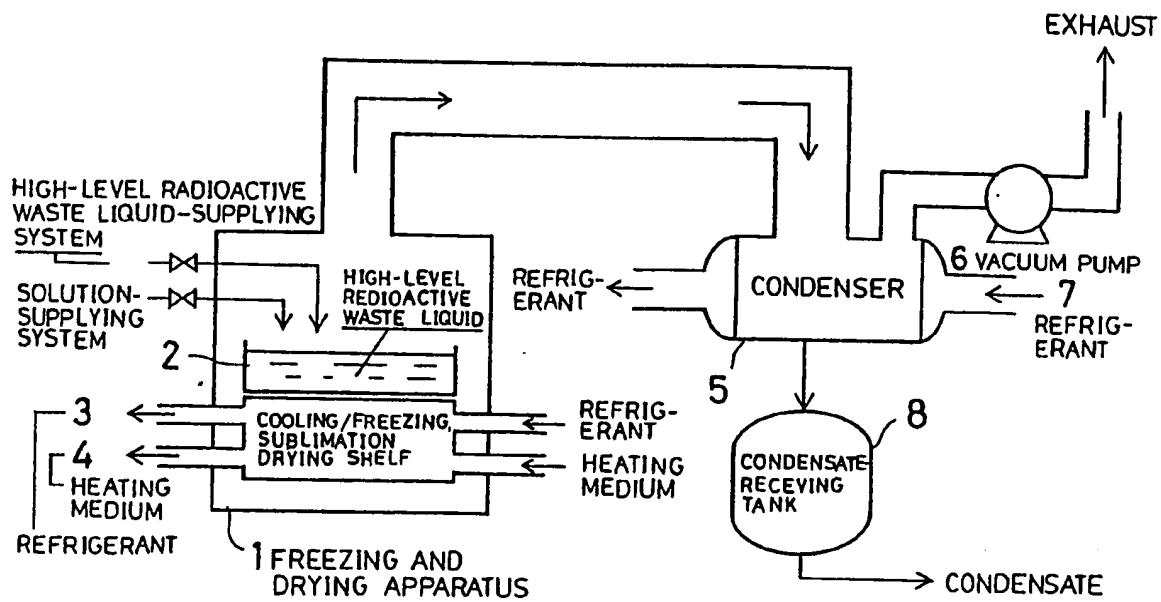
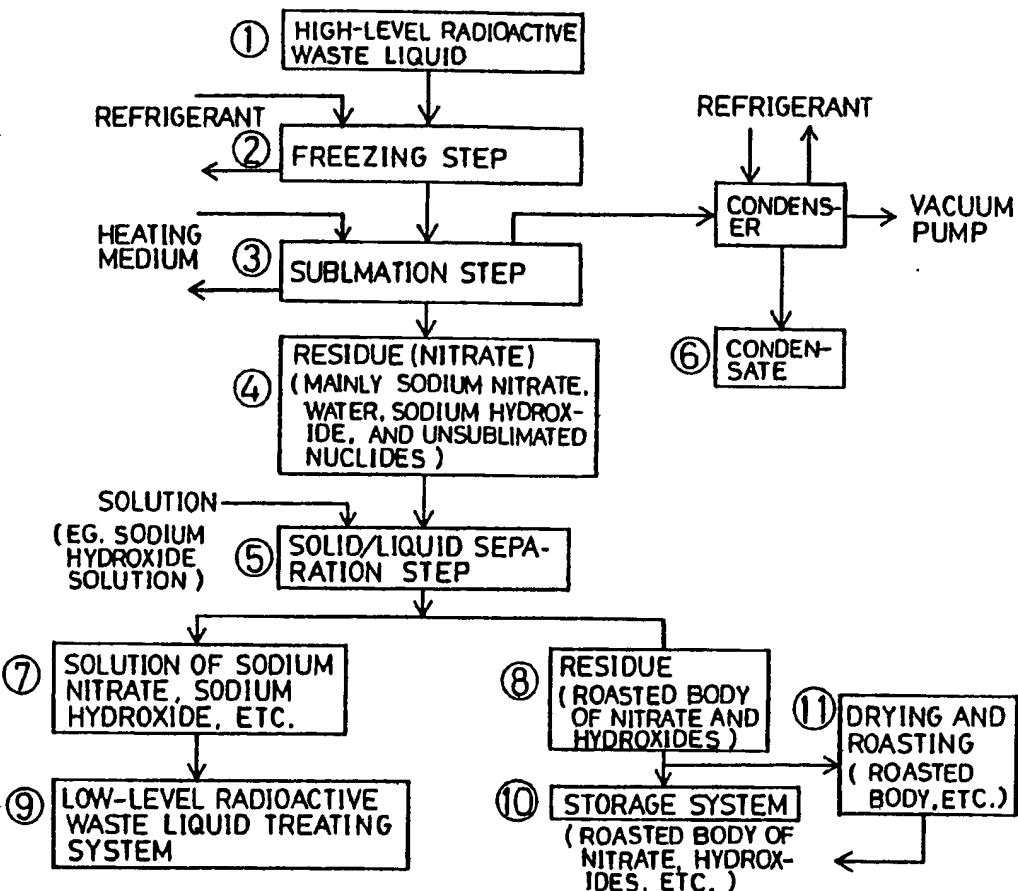


FIG. 2





European Patent  
Office

## EUROPEAN SEARCH REPORT

Application Number

EP 89 30 6156

DOCUMENTS CONSIDERED TO BE RELEVANT			
Category	Citation of document with indication, where appropriate, of relevant passages	Relevant to claim	CLASSIFICATION OF THE APPLICATION (Int. Cl. 4)
X	GB-A-2 178 588 (DORYOKURO) * Claims 1,3,6,7 * ----	1-3	G 21 F 9/08
A	FR-A-1 333 345 (LEYBOLD) * Abstracts 1,4,5 * -----	1,2,5	
TECHNICAL FIELDS SEARCHED (Int. Cl.4)			
G 21 F			
The present search report has been drawn up for all claims			
Place of search	Date of completion of the search	Examiner	
THE HAGUE	19-07-1989	NICOLAS H.J.F.	
CATEGORY OF CITED DOCUMENTS		T : theory or principle underlying the invention E : earlier patent document, but published on, or after the filing date D : document cited in the application L : document cited for other reasons ..... & : member of the same patent family, corresponding document	
X : particularly relevant if taken alone Y : particularly relevant if combined with another document of the same category A : technological background O : non-written disclosure P : intermediate document			